

# ***U.S. EPR Fuel/Core Design***

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# Core Design

## ➤ Core Size

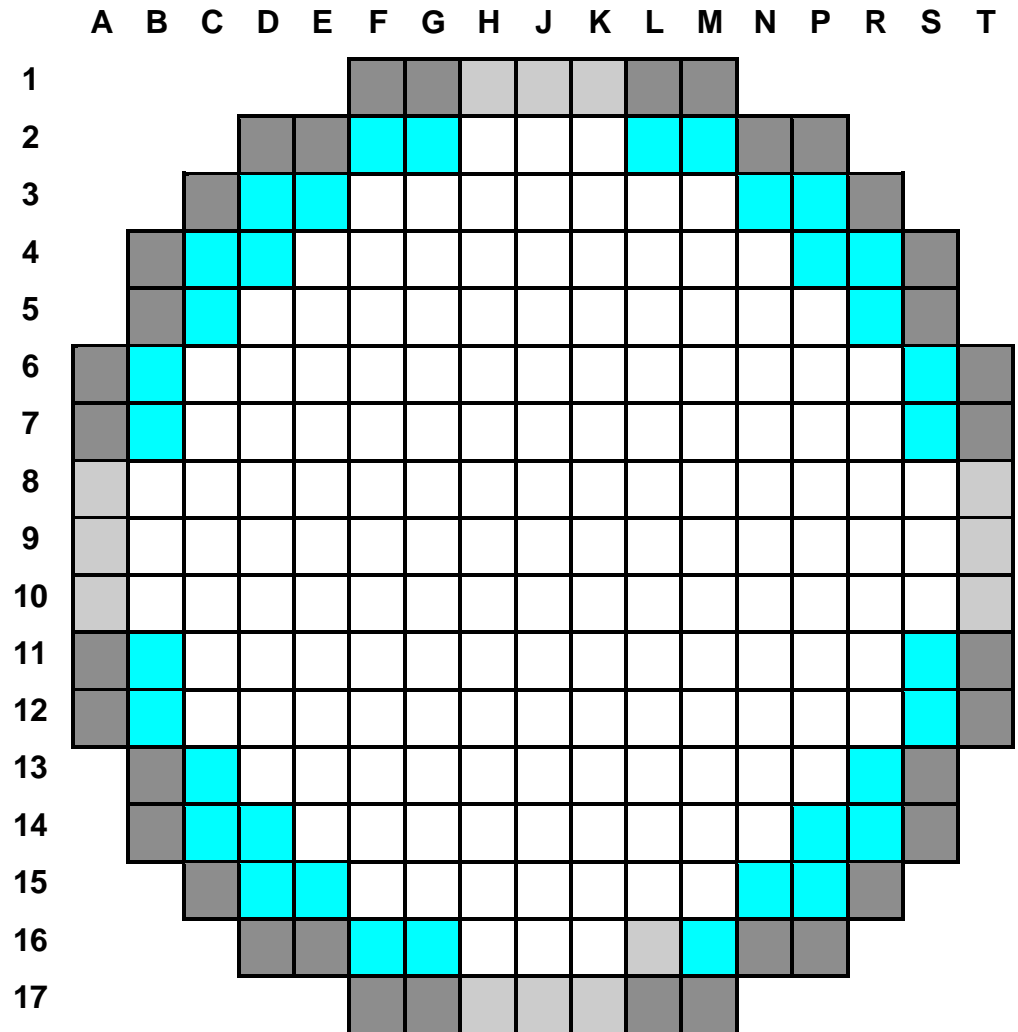
- ◆ 241 fuel assemblies
- ◆ active fuel length 4.2 m (  $\approx$  13' 9"
- ◆ Allows long fuel cycles (up to 2 years) with a good fuel cycle economy
- ◆ Low Linear Heat Rate
  - $\approx$  5.1 kW/ft (for 4590 MWth)
  - High Radial Peaking Factor  $F_{\Delta h} \leq 1.63$   $F_q < \approx 2.7$
  - Full low leakage loading pattern capability
- ◆ Design margin preserves future flexibility

# From The 900 Mwe 3-loop Plant To EPR

From the 3-loop to the  
4-loop 1300 MWe  
+36 assemblies

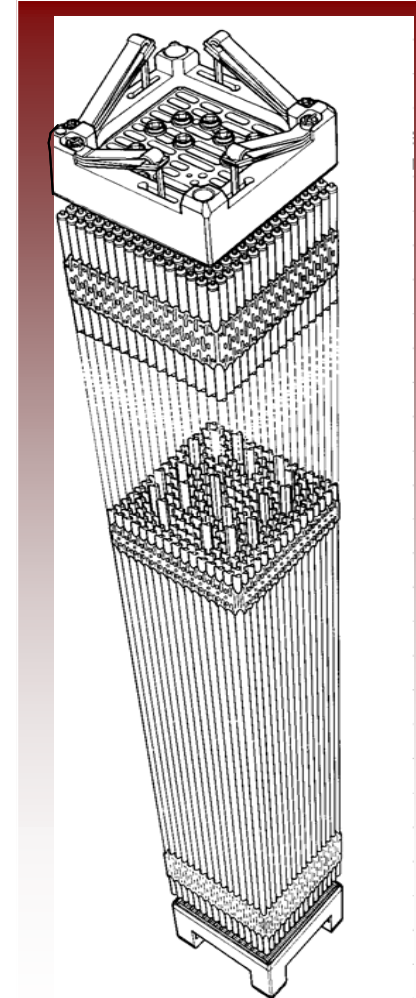
From N4 to EPR  
+36 assemblies

Type of plant	Nb of fuel assy	
3-loop	157	
4-loop 1300 MWe	193	
4-loop N4	205	
EPR	241	

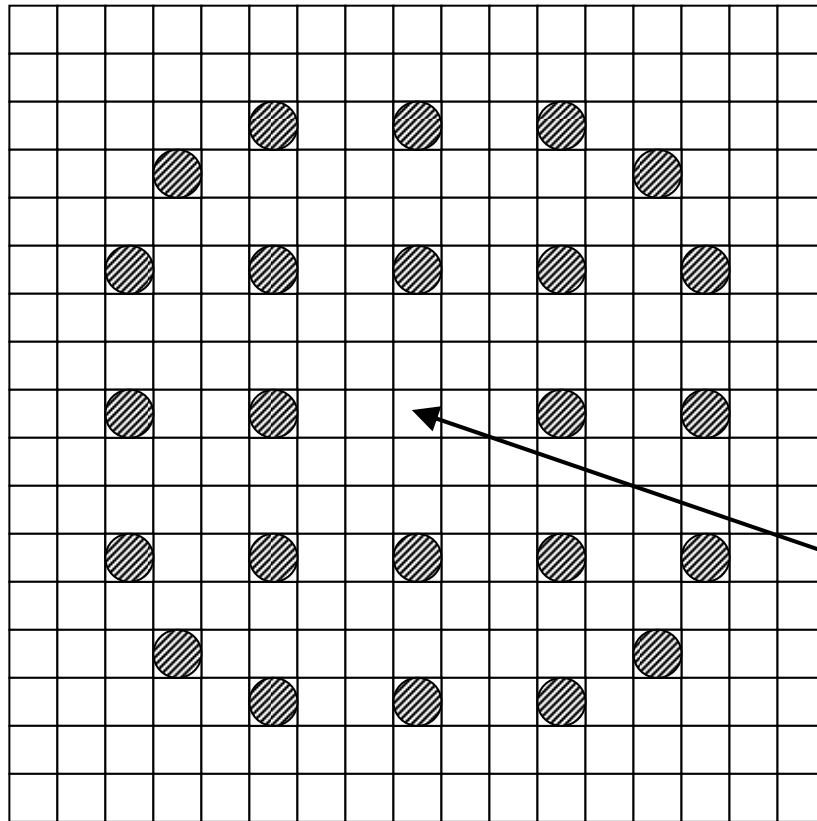


# Fuel Assembly

- 17x17
- Typical Pitch-to-Diameter Ratio
- 265 Fuel Rods Per Assembly
- M5 Cladding
- Heated Length (13.78') Similar to N4 (14')
- M5 HTP Mixing Vane Grids (8)
- Anti-Debris Lower End Fitting
- Significant Design Margins
- Variable  $Gd_2O_3$  content up to 8%
- $U_{235}$  enrichment up to 5%
- MOX Compatible



# 17x17 ASSEMBLY

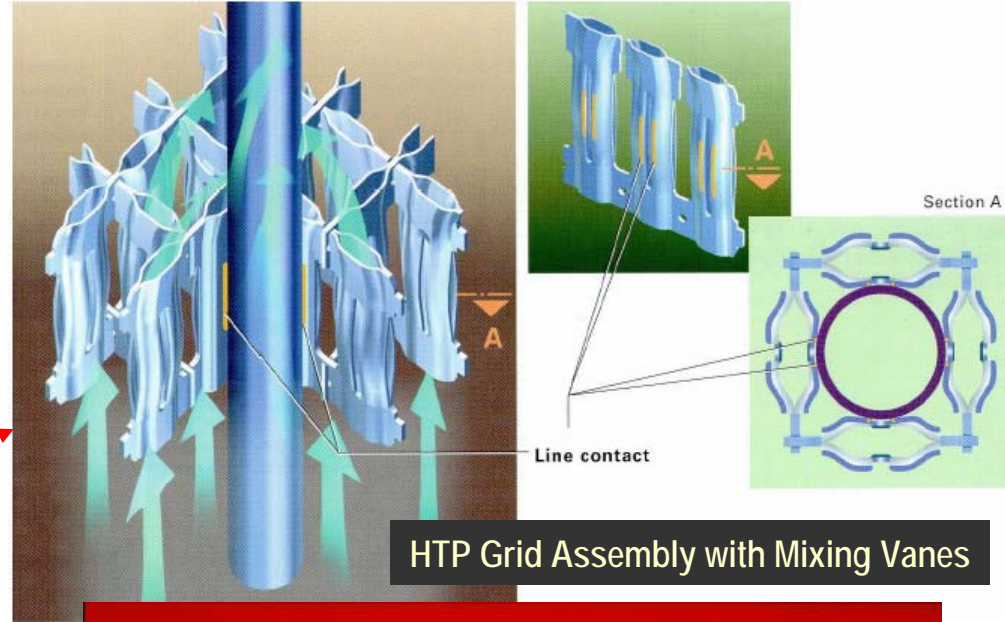
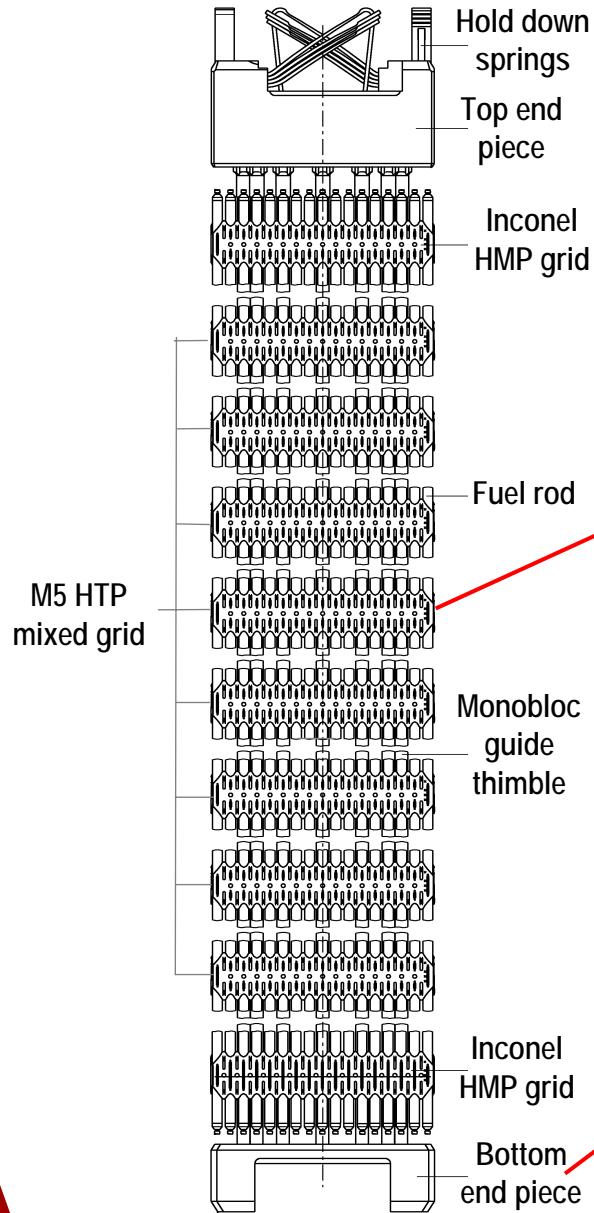


□ Fuel Rod

▨ Guide Tube  
for RCCA absorber rod or  
instrumentation thimble

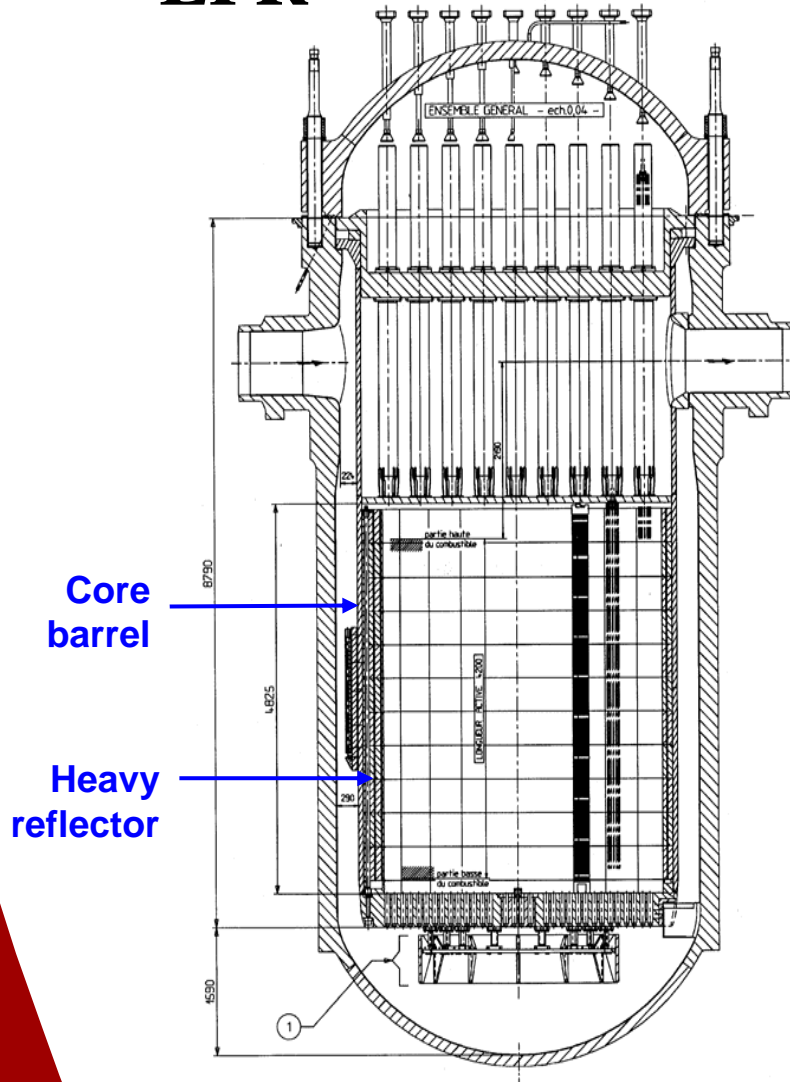
No central  
instrumentation  
guide tube

# HTP Fuel Assembly

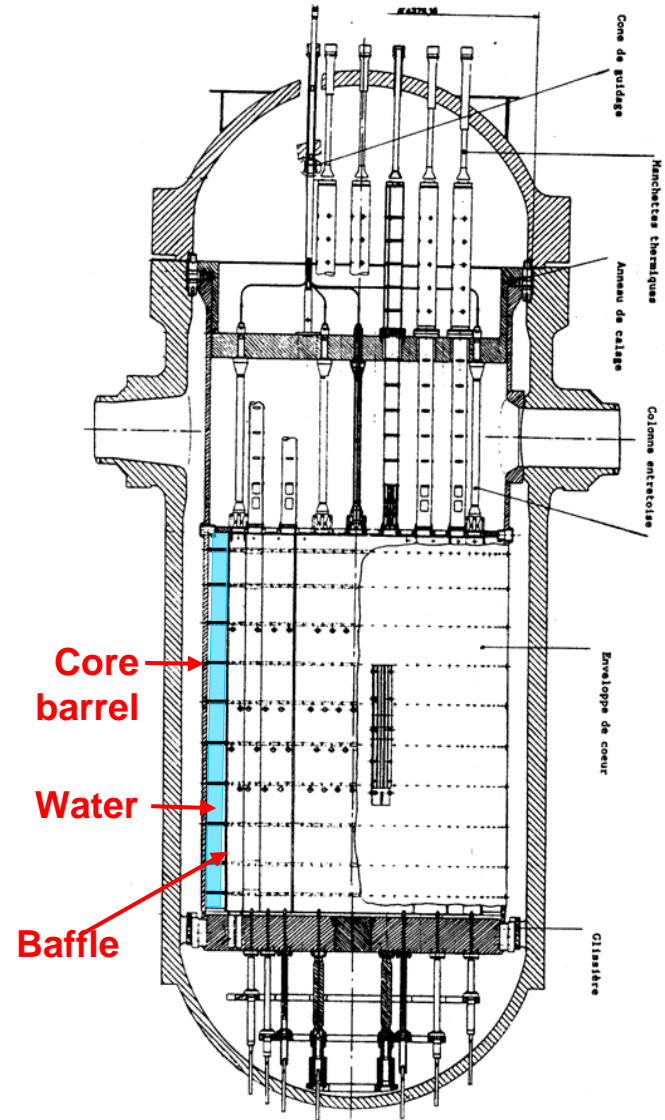


# The Heavy Reflector in the Reactor Pressure Vessel

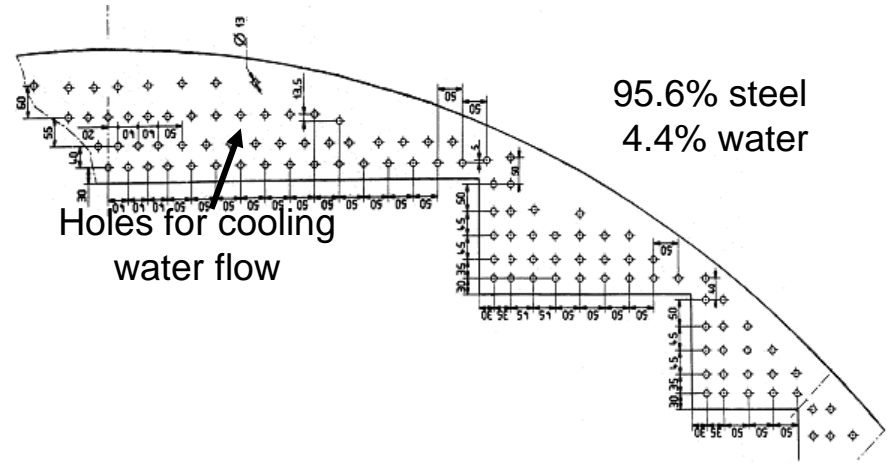
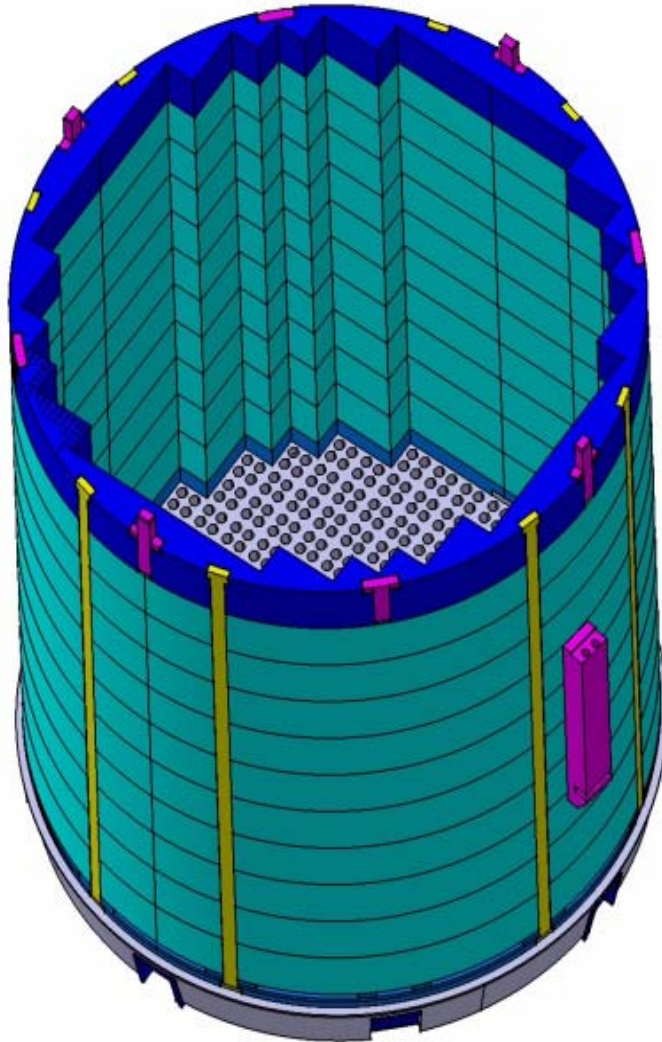
**EPR**



**N4**



# The Heavy Reflector



- **Good mechanical behavior**
  - ◆ No bolts in high fluence region.
  - ◆ No welds.
  - ◆ Reduced LOCA forces
  - ◆ No "baffle jetting"
  - ◆ Removable
- **Reduces neutron leakage**
  - ◆ Fuel cycle cost saving : 1%
  - ◆ Smaller pressure vessel diameter for a given limiting vessel flux level



# *Flexible Cycle Length And Fuel Management Strategy*

- Special attention was paid to flexibility with respect to possible reload enrichments, fuel types, core designs and discharge burnup
  - ◆ Fuel cycle length of 18 months taken as design basis. Also considered:
    - 24 months fuel cycle length
    - In/Out and Out/In loading pattern
  - ◆ Maximum enrichment of 5%
  - ◆ Average discharge burnup consistent with licensed limits (62 GWD/Mt), but consider future stretch
  - ◆ Stretch out or coast-down operations at end of cycle
- Maneuverability capability ensured for the different type of fuel loading
- Capability for Plutonium recycling (MOX assemblies)

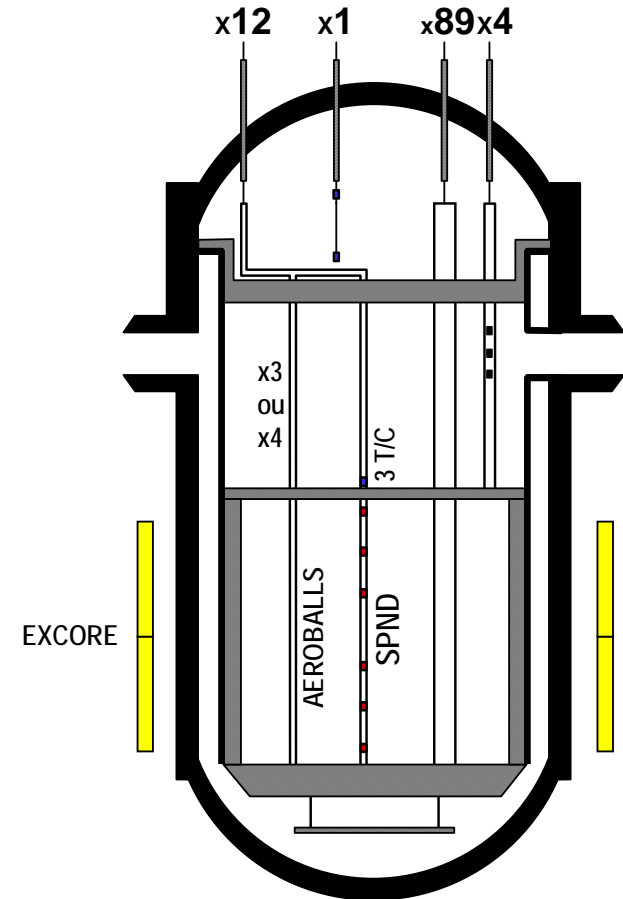
***80 FA Feed For 18 Month Equilibrium Cycle At 4.9 w/o***

# Fuel Management

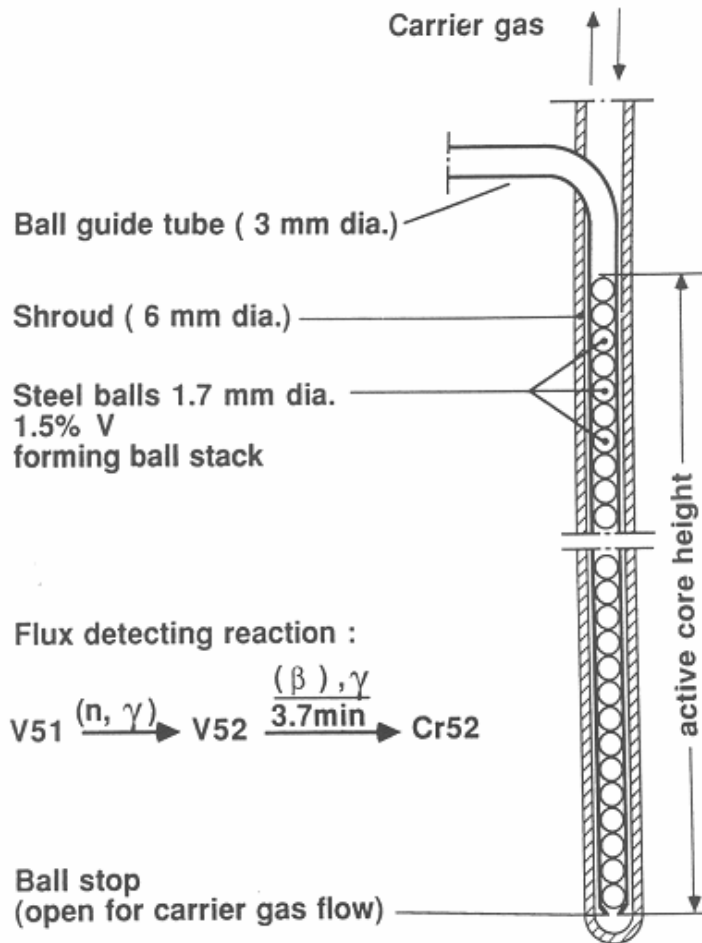
- **Moderator coefficient always negative**
  - ◆ bounding conditions : zero power ; all rods out
  - ◆ the amount of Gd poison is set in order to limit boron concentration
  
- **Hot channel enthalpy rise factor restricted to avoid fuel corrosion**
  - ◆ 1.63 during Basic Design studies
  
- **Core subcritical after partial cooldown (520 F)**
  
- **Enriched B<sub>10</sub> for the core reactivity control**
  
- **Accommodate MOX**

# Nuclear Instrumentation

- In-core instrumentation based on German design and experience
  - ◆ Aeroballs
    - Reference instrumentation for power distribution measurement
  - ◆ Self Powered Neutron Detectors
    - Fixed incore instrumentation for online core monitoring (surveillance and protection)
    - Calibration based on Aeroball measurements
- Excure neutron detectors
  - ◆ Source, intermediate and power ranges
  - ◆ Protection against fast reactivity increase



# Aeroball system



- **Aeroball measurement system (AMS)**
  - ◆ activity measurement of a ball (steel+vanadium) stack after insertion into fingers in the assembly.
- **Reference Instrumentation**
  - ◆ Periodical neutron flux map for checking the core conformity and calibrating the fixed incore nuclear instrumentation
- **Short measuring time**
  - ◆ Very short activation time => flux mapping in 15 minutes
  - ◆ No impact on plant maneuverability : no need of xenon equilibrium when performing a flux map

# Fixed incore instrumentation







- Fixed incore instrumentation based on cobalt SPNDs
  - ◆ 12 fingers with 6 Self Powered Neutron Detectors each
- Signals used for control, surveillance and protection of the core
  - ◆ on line calculation
    - Axial offset \*
    - Hot spot factor (Fq) and High Linear Power Density (HLPD)
    - Minimum DNBR
  - ◆ Periodic calibration every 15 days

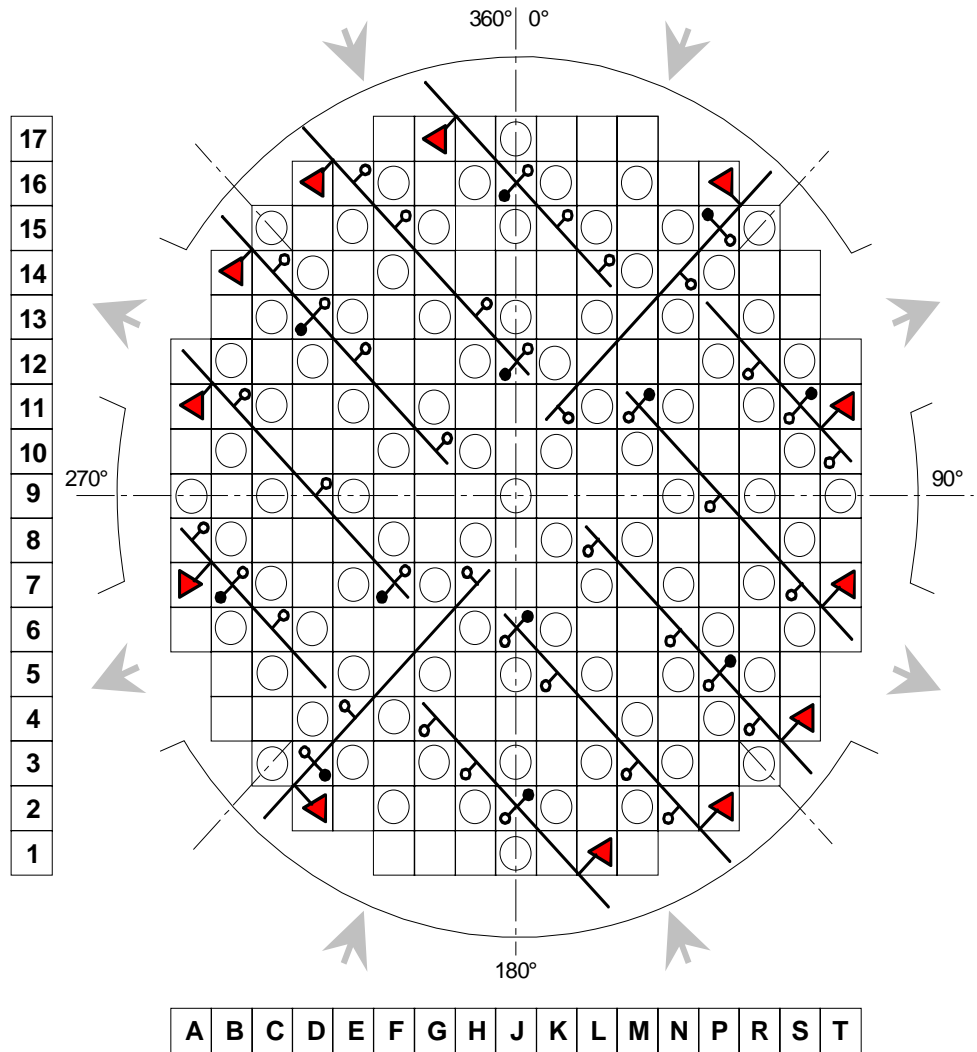
\* Core Axial Offset : 
$$AO = \frac{P_T - P_B}{P_T + P_B}$$

$P_T$  and  $P_B$  : Power generated in the top and bottom halves of the core

# In-core Instrumentation Pattern

Detectors are located only in unrodded fuel assemblies  
 No significant impact on the flux map accuracy

-  241 FUEL ASSEMBLIES
-  40 AEROBALL PROBES
-  12 RPV INSTRUMENTATION NOZZLES WITH INSTRUMENTATION LANCES
-  89 CONTROL ASSEMBLIES
-  12 FINGERS WITH POWER DENSITY DETECTORS (SPNDs) AND THERMOCOUPLES
-  12 LANCE YOKES



# Core Thermal Hydraulic Monitoring

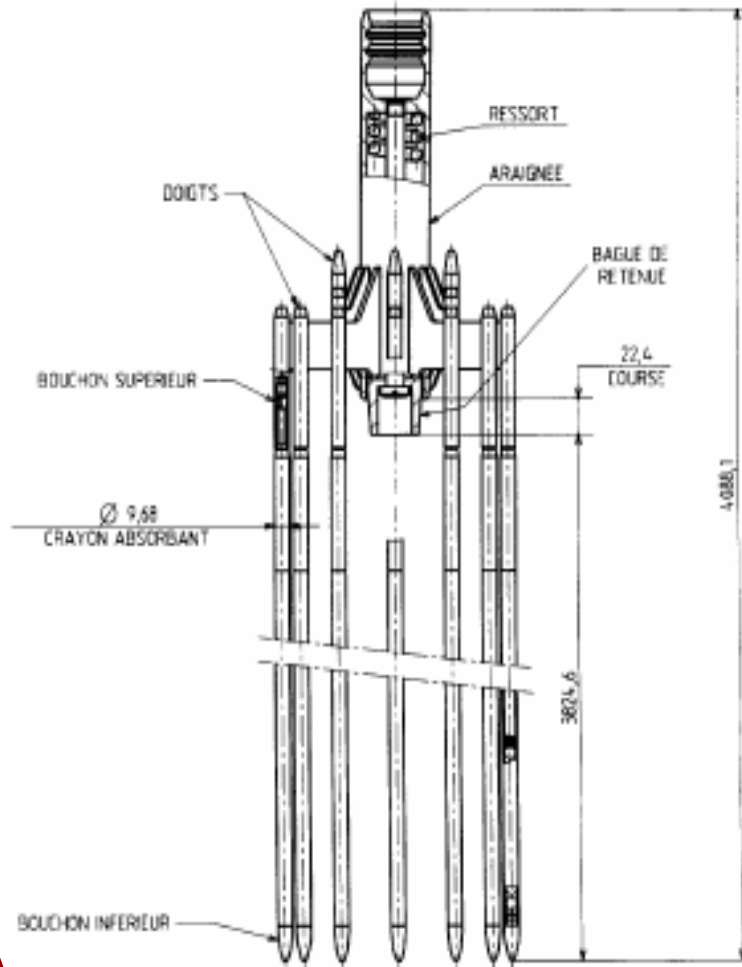
- **Minimum DNBR is calculated by an algorithm accounting for :**
  - ◆ **Inlet temperature,**
  - ◆ **Pressure,**
  - ◆ **Flow rate (RC Pumps speed)**
  - ◆ **Power level**
  - ◆ **Radial distribution ( $F\Delta h$ )**
  - ◆ **Hot channel axial power distribution**
    - **The axial hot channel power distribution is reconstructed from the six signals of the SPND thimble.**
  - ◆ **Tracking Error**
    - **Account for the accuracy of the relationship between the SPND signals and the parameter ( $Fq$ ,  $F\Delta h$ ) in the monitoring zone**
  
- **Core protection against low DNBR**
  - ◆ **Surveillance/Limitation**
    - **1) Stop control rod withdrawal and load increase**
    - **2) Power reduction: rod insertion (partial trip)**
  - ◆ **Protection**
    - **Reactor trip**

# Core Reactivity Control Principles

<b>Soluble Boron</b> - CVCS - EBS	<b>Fuel burnup compensation</b> <b>Xenon compensation</b> <b>Ensure subcriticality at cold state</b>	<b>Enriched B<sub>10</sub> keeps C<sub>B</sub> &lt; 1400 ppm (HFP, BOC)</b>
<b>Control rods</b>  <b>89 black rods</b>	<b>Control</b> - Reactor coolant T°avg - Axial Offset  <b>Ensure subcriticality for core safe shutdown (260 °C)</b>	<b>Control of rod bank insertion</b>  <b>Partial trip</b>  <b>Reactor trip</b>
<b>Burnable absorbers (Gd<sub>2</sub>O<sub>3</sub>)</b>	<b>Reduce boron conc at BOC</b>  <b>Limit radial power peaking factor</b>	<b>Moderator T° coef &lt; 0</b>  <b>FΔh &lt; FΔh limit</b>



# Rod Cluster Control Assembly (RCCA)



- Only "black rods"
  - ◆ 24 hybrid control rodlets
  - ◆ upper part B4C natural boron
  - ◆ lower part Ag/In/Cd
  - ◆ U.S. EPR – AIC RCCAs
  
- Control rod drive mechanism with magnetically operated jack
  - ◆ 1 step = 10 mm

# RCCA Pattern

- **89 RCCA for maximizing the shutdown margin**
- **53 for shutdown (N)**
- **36 for control**  
**9 banks of 4 rods symmetrically located <sup>270°</sup> (PA to PI)**  
**The 4 rods move at the same time with the same signal**
- **Assignment of bank to control groups can be changed during the fuel cycle**

